

1 Overview on the Progress of the Conceptual Studies 2 of a Gamma Ray Spectrometer Instrument for DEMO

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15 **ABSTRACT:** The future DEMO tokamak will be equipped with a suite of diagnostics which will operate as
16 sensors to monitor and control the position and operation parameters of DT plasmas. Among the suite of
17 sensors, an integrated neutron and gamma-ray diagnostic system is also studied to verify its capability and
18 performance in detecting possible DEMO plasma position variations and contribute to the feedback
19 system in maintaining DEMO DT plasma in stable conditions. This work describes the progress of the
20 conceptual study of the gamma-ray diagnostic for DEMO reactor performed during the first Work-
21 Package contract 2015-2020.

22 The reaction of interest for this Gamma-Ray Spectrometer Instrument (GRSI) consists of $D(T, \gamma)^5\text{He}$ with
23 the emission of 16.63 MeV γ rays. Due to DEMO tokamak design constraints, the gamma and neutron
24 diagnostics are integrated, both featuring multi-line of sight (camera type), viewing DEMO plasma
25 radially with vertical (12) and horizontal (13) viewing lines to diagnose the γ and neutron emission from
26 the DT plasma poloidal section.

27 The GRSI design is based on the investigation of the reaction cross sections, on the calculations
28 performed with GENESIS and MCNP simulation codes and on the physics and geometry constraints of
29 the integrated instrument. GRSI features long collimators which diameters are constrained by the neutron
30 flux at the neutron detectors of the Radial Neutron Camera (RNC) system placed in front, which are key
31 to control DEMO DT plasma position. For these reasons, only few GRSI parameters can be
32 independently selected to optimize its performance. Among these, the choice of the collimator diameters
33 at the back side of the neutron detector box up to the GRSI detector, the use of LiH neutron attenuators in
34 front of the GRSI detectors, the GRSI detector material and shielding.

35 The GRSI detector is based on commercial $\text{LaBr}_3(\text{Ce})$ inorganic scintillating crystal coupled with a
36 photomultiplier tube or a Silicon photomultiplier. They are designed to operate at high count rate
37 although GRSI geometry constraints severely impact on this feature. The GRSI can also provide an
38 independent assessment of DEMO DT fusion power and T burning.

39 **KEYWORDS:** DT fusion plasmas; DEMO diagnostics; high-energy gamma-ray spectroscopy;
40 $\text{LaBr}_3(\text{Ce})$ inorganic scintillating detector.

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58 1. Introduction

59 The design of the diagnostics for the position control of DEMO Deuterium-Tritium (DT)
60 plasmas completed its first six-year long design contract and underwent an external panel
61 review [1][2][3]. The Gamma-Ray Spectrometer Instrument (GRSI) is one of these set of
62 diagnostics. It aims at the spectroscopic measurements of DT gamma-ray emission of $D(T,\gamma)^5\text{He}$
63 and $T(p,\gamma)^4\text{He}$ reactions of 16.63 MeV ($DT\gamma$) and 19.81 MeV ($Tp\gamma$), respectively, to be able to
64 monitor and control the DT plasma position during the DEMO discharge with 10 ms time
65 resolution [2][3].

66 The GRSI is thought to be installed at the back side of the Neutron Vertical and Horizontal
67 Cameras (VNC and HNC, respectively) [4], outside DEMO biological shielding with very long
68 collimators. With respect to the initial studies [5], new aspects of the GRSI design have been
69 considered [6], namely:

- 70 1) the count rate sensitivity to detector and collimator geometry;
- 71 2) the GRSI capability and accuracy for DEMO plasma position control;
- 72 3) $DT\gamma$ and DTn cross-talk in nearby GRSI detectors;
- 73 4) effects of the shielding materials on GRSI performance;
- 74 5) capability in assessing the impurity concentrations in DEMO DT plasmas by measuring
75 their X-ray emission;
- 76 6) effects of the accurate knowledge of $DT\gamma/DTn$ branching ratio (BR) on the GRSI per-
77 formance;
- 78 7) capability of assessing DEMO DT fusion power via γ ray measurements;
- 79 8) capability of assessing the density nT/nD ratio.

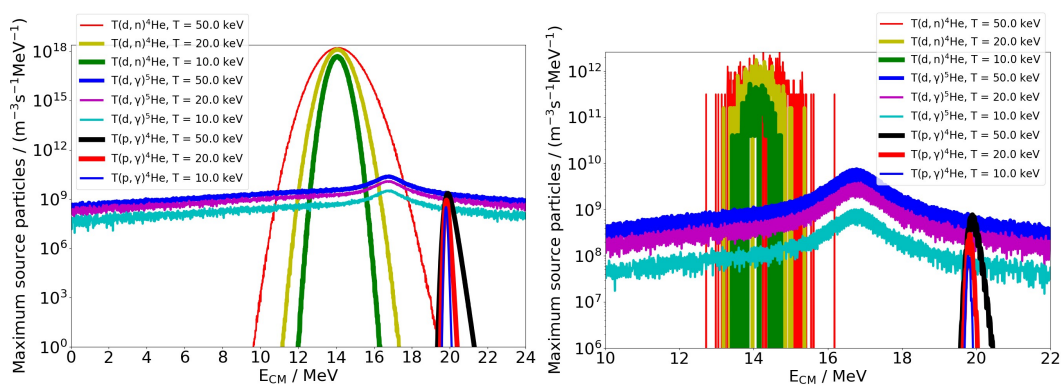
80 All these aspects are reported and discussed in the following section which provide an overview
 81 on the status of the project and on the choices for the further development of the GRSI design.

82 2. Progress on GRSI design to optimize its performance for DEMO plasma 83 position control

84 2.1 GRSI detector geometry and collimators

85 The Radial Neutron Camera (RNC) is designed with fan-shaped collimators, 12 for the Vertical
 86 Neutron Camera (VNC) and 13 for the Horizontal Neutron Camera (HNC). Each RNC
 87 collimator is designed as a very long cylinder in front of the RNC neutron detector box which
 88 sees directly DEMO vessel and its incoming neutron flux. The GRSI is planned to be installed
 89 at the backside of the RNC making use of the same lines of sight. The GRSI collimators are
 90 defined as a cylinder from the backside of the RNC neutron detector box to the front of GRSI
 91 detector. This design constraint has been investigated in terms of GRSI detector and collimator
 92 diameters in order for the GRSI to reach count rates compatible with the diagnosis and control
 93 of DEMO plasma position. Two geometries of DT γ spectrometers based on LaBr₃(Ce) inorganic
 94 scintillating material [7] have been considered to assess their efficiency: commercial 7.62 cm x
 95 15.24 cm (3"x6" diameter x length) and 9.65 cm x 24.64 cm (3.8"x9.7"). A Monte Carlo model
 96 based on MCNP code [8] has been setup to evaluate their detection efficiency for incoming DT γ
 97 flux of 16.63 MeV: These result in 93 % and 99 %, respectively. The RNC neutron fluxes at the
 98 various neutron detector positions have been used as input to assess the corresponding γ count
 99 rate based on the assessment of BR in literature and on GRSI geometry. At the beginning of the
 100 GRSI design in 2015, the BR was studied considering the available literature reporting of
 101 experimental assessments performed in 1980s and 1990s for various centre of mass DT
 102 energies. Each assessment features its own experimental uncertainties. To find the BR value
 103 correspondent to DEMO burning plasma conditions, a fit of all these BR values has been
 104 performed and a conservative extrapolated BR value of $2.07 \cdot 10^{-6}$ obtained [5].

105 The simulation code GENESIS [9] has been used to define the input 14.01 MeV DT neutron
 106 (DTn) spectrum for DEMO DT plasmas at equilibrium temperatures of 10 keV, 20 keV and 50
 107 keV. The DT γ spectrum has been then obtained considering the DTn cross section and the
 108 DT γ /DTn branching ratio $BR = 2.07 \cdot 10^{-6}$ [5], while the T γ spectrum directly from T γ cross
 109 sections [6].



110
 111 **Figure 1.** On the left panel, incoming DTn, DT γ and T γ spectra distributions for DEMO thermal
 112 plasmas at 10 keV, 20 keV and 50 keV. On the right panel, the corresponding spectra at the GRSI
 113 detector position after travelling through 120.0 cm long LiH neutron attenuator.

114 A key component of the GRSI consists of a LiH neutron attenuator positioned in front of the
 115 GRSI detectors to suppress the incoming DTn flux. Based on MCNP calculation, 120.0 cm long

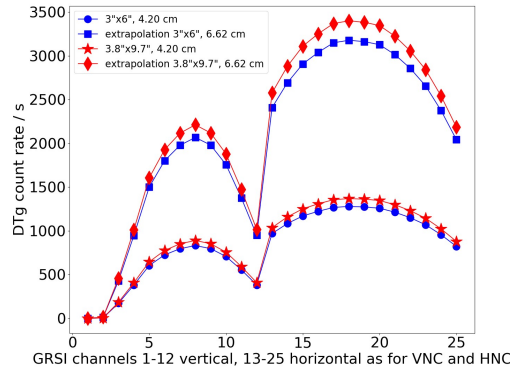
116 LiH cylinder featuring attenuation factors of 10^6 to DTn and 0.3 to DT γ has been found suited
 117 for the GRSI designed. Figure 1 shows the effects of LiH neutron attenuator for DT plasmas at
 118 different equilibrium temperatures.

119 The impact of the RNC and GRSI collimator diameters on the DT γ GRSI measurement have
 120 been evaluated in terms of DT γ count rates. **Different diameters of the GRSI collimators,**
 121 **namely, 1.10 cm, 2.20 cm, 4.20 cm and 6.62 cm, have been studied to assess the GRSI count**
 122 **rate capability.** In Table 1 the maximum count rates at the GRSI central channel detector of the
 123 vertical and horizontal arrays are reported for detector dimensions of 3"x6" and 3.8"x9.7". With
 124 6.62 cm diameter collimators, GRSI would reach 2.5 times higher count rates of pure DT γ
 125 events with respect to 4.20 cm. The edge channels features about half the rate of the central
 126 channels as displayed in Figure 2.

127 Table 1. Maximum count rates (in kCounts/s) assessed for the vertical and horizontal GRSI central
 128 channel for collimator diameters of 2.20 cm, 4.20 cm and 6.62 cm and detector dimensions of 3"x6" and
 129 3.8"x9.7".

GRSI detector	GRSI V 2.20 cm	GRSI H 2.20 cm	GRSI V 4.20 cm	GRSI H 4.20 cm	GRSI V 6.62 cm	GRSI H 6.62 cm
3"x6"	0.32	0.21	1.28	0.83	3.32	2.12
3.8"x9.7"	0.34	0.22	1.37	0.89	3.55	2.27

130 Now, considering the effects of the background (the larger the detector volume the larger its
 131 sensitivity to the 4π distributed neutron and γ background), the commercial costs and the
 132 instrument performance, the GRSI for DEMO can implement 3"x6" LaBr₃(Ce) scintillators
 133 similarly to the ones already installed and operational at the Joint European Torus (JET,
 134 Culham, UK) [10] and those defined for the Radial Gamma-Ray Spectrometers (RGRS) system
 135 for ITER [11].

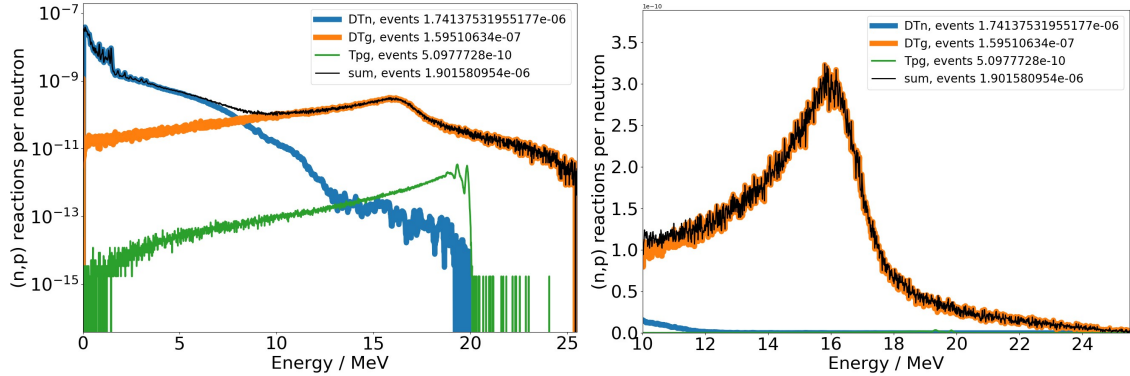


136
 137 **Figure 2.** Comparison of the GRSI DT γ count rates measured in each vertical (1-12) and horizontal (13-
 138 25) channel for 4.20 and 6.62 cm diameters and for 3"x6" and 3.8"x9.7" LaBr₃(Ce) scintillators.

139 2.2 Assessment of the GRSI measured 16.63 MeV DT gamma-ray spectrum

140 The effects of the DTn induced background on the direct DT γ spectrum has been investigated
 141 assuming GRSI 4.20 cm diameter collimators equipped with 120.0 cm long LiH neutron
 142 attenuators in front of the 3"x6" LaBr₃(Ce) scintillators. Figure 3 displays the simulated GRSI
 143 measured γ spectrum. About 93.5 % of the total events in the spectrum fall below 10 MeV
 144 which 99.9 % are due to DTn induced background events. 98.6 % of 16.63 MeV DT γ events are
 145 above 10 MeV with the 19.81 MeV T $\rho\gamma$ spectrum depressed by a factor 200. These results
 146 confirm GRSI DT γ detection as mostly background-free.

147 Given the characteristic and quality of the GRSI measurements, a 3"x6" LaBr₃(Ce) scintillator
 148 may be capable of 80 genuine DT γ counts per 100 ms for DEMO position control. This does not
 149 match DEMO requirements on time resolution (10 ms) and accuracy but, given the GRSI
 150 installation constraints and their impact on its performance, the GRSI can still be used to
 151 benchmark the RNC diagnostic measurements and to provide physics information on more
 152 relaxed time resolutions.



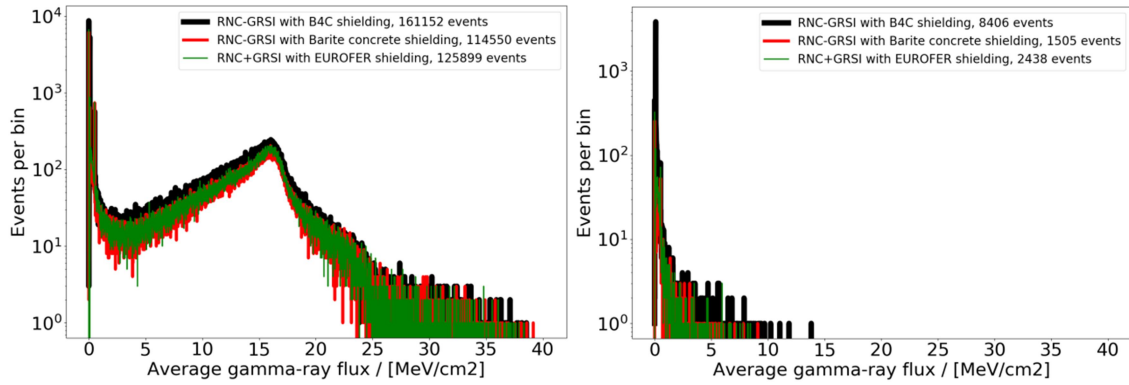
153
 154 **Figure 3.** On the left panel, comparison of DTn, DT γ and Tp γ contribution to the simulated GRSI
 155 measured gamma-ray spectrum in log scale. On the right panel, the same in linear scale (times $1 \cdot 10^{-10}$) for
 156 deposited energies in LaBr₃(Ce) scintillator larger than 10 MeV.

157 2.3 Assessment of the GRSI gamma-ray and neutron cross-talk in nearby detectors

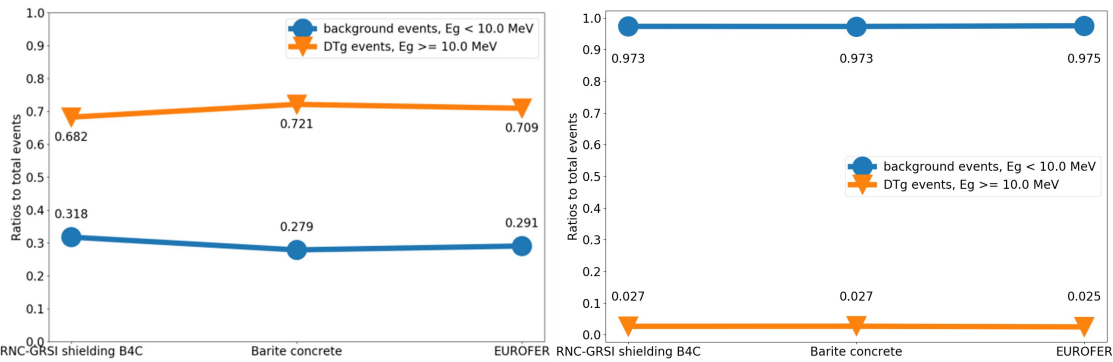
158 Being the GRSI, as the RNC, a multi-line of sight instrument, the cross-talk of neighboring
 159 channels needs to be taken into account. The scattered events in one GRSI channel/detector into
 160 the neighboring one hampers the accuracy of the plasma position information available **from the**
 161 **measurement of each** individual channel/detector and on **the DT γ profile** reconstruction as a
 162 whole. MCNP calculations have been carried out for incoming DT γ and DTn along one specific
 163 line of sight (4.20 cm diameter) into its 3"x6" scintillator to assess the number of events detect-
 164 ed also in the nearby similar detector. The shielding is assumed to be made of B₄C as in the
 165 RNC design [4]: **Natural Boron is considered with density of 2.5 g/cm³ and approximately 20%**
 166 **of ¹⁰B and 80% of ¹¹B.** The cross-talk induced by DT γ results in an exponentially distributed
 167 energy spectrum up to 6 MeV and a factor $5.22 \cdot 10^{-2}$ lower with respect to the direct spectrum.
 168 The cross-talk induced by neutrons is **thus** negligible [6].

169 2.4 Assessment of the GRSI shielding material on measured gamma-rays

170 Three different materials (B₄C, barite concrete and EUROFER) have been investigated for the
 171 GRSI shielding modeled in MCNP [8] and considering the simulated measurement of direct
 172 DT γ , induced cross-talk and background events. As shown in Figure 4 on the left, the B₄C
 173 shielding (black line) seems to feature a larger number of simulated events within 5-15 MeV
 174 range for fluxes lower than 15 MeV/cm² in the direct measured spectrum and larger than 3
 175 MeV/cm² in the cross-talk spectrum (on the right). Now the impact of the background caused by
 176 the shielding material can be quantified considering the number of good events above 10 MeV
 177 in the spectra. Figure 5 illustrates the fractions of direct and background events below and above
 178 10 MeV energy threshold depending on the shielding material. Barite concrete shielding
 179 features the largest fraction of good events above 10 MeV, 5.4 % better than B₄C, and the
 180 lowest background.



181
182 **Figure 4.** Comparison of the direct (left) and cross-talk (right) DT γ simulated spectra measured along two
183 neighbouring lines of sight of the GRSI shielded with B₄C, barite concrete or EUROFER.



184
185 **Figure 5.** Comparison of the fractions of good events and background in the direct (left) and cross-talk
186 (right) DT γ simulated spectrum measured along two neighbouring lines of sight of the GRSI shielded
187 with B₄C, barite concrete or EUROFER.

188 2.5 GRSI capability of measuring impurity concentration in DEMO DT plasmas

189 DEMO DT plasmas will feature impurities of W, Xe, Kr, Ar, Fe, all at very low concentrations.
190 The cross-sections and energies of their X-ray emission has been studied in order to verify the
191 GRSI capability of detecting it. These impurities emit several low energy (about 10² keV) X
192 rays from nuclear excited states with reactions cross sections of about 1 barn (10⁻²⁸ m²). In case
193 of Tungsten, the typical plasma electron density is about 10²⁰ electrons/m³ with Tungsten
194 concentration ratios in the tokamak vessel of n_W/n_e = 10⁻⁴. In these conditions, the low Tungsten
195 concentration (n_W = 10¹⁶ W atoms/m³) and X-ray energy emissions (about 10² keV) result in X-
196 rays which may be detected in the region of LaBr₃(Ce) intrinsic activity (up to 3 MeV, see
197 Figure 11) and not clearly recognizable [7]. This is also true for the other impurities
198 investigated.

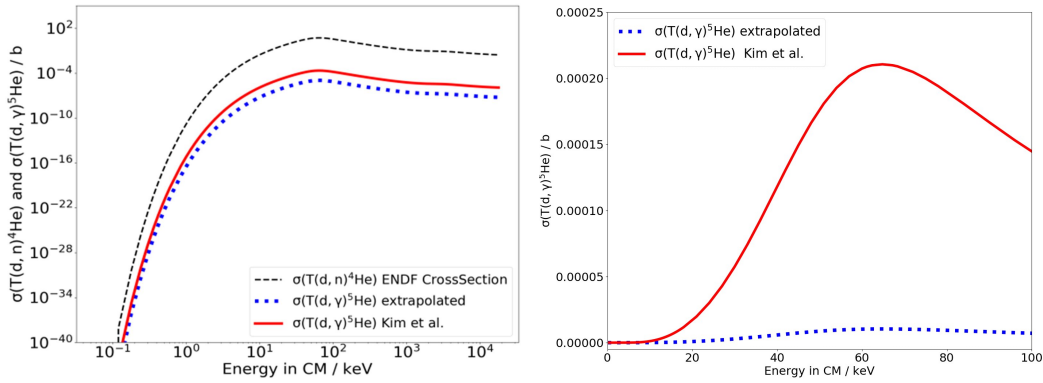
199 In fact the detector crystal for the GRSI project made of LaBr₃(Ce) has been optimized for the
200 diagnosis of 16.63 MeV DT γ rays. If the goal of the measurement would be the diagnosis of the
201 X-ray emission of plasma impurities, a dedicated diagnostic with different geometry constraints
202 and detector material and/or technology (i.e., gas electron multiplier GEM detectors) would
203 have to be considered.

204 Also the diagnosis of fusion reactions involving ³He has been considered in the GRSI
205 assessment. ³He spatial distribution, accumulation, "thermal" and "high energy" fraction with
206 few seconds time resolution are of interest to assess the DT burning efficiency: **In DT plasmas
207 at a certain temperature**, ³He is produced from DD reactions with 50 % branching ratio and two
208 orders of magnitude lower cross section than DT. In its slowing down motion and ion/electron

209 collisions in DT plasmas, ^3He may react with D giving rise to ^5Li and 16.60 MeV γ rays (10^{-3}
 210 branching ratio). As this reaction results from a low probability two-step process with energy
 211 similar to the direct DT γ of interest, the ^3He signature is difficult to be diagnosed with the GRSI.

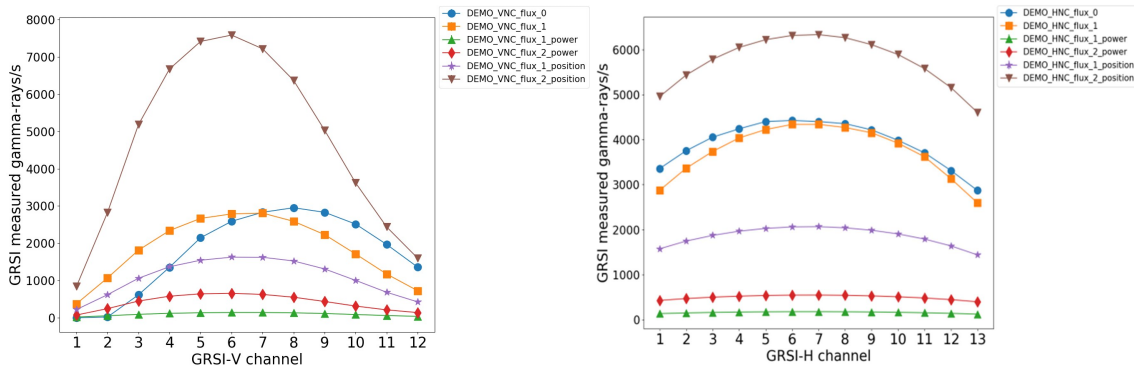
212 2.6 Effects of the DT γ /DTn branching ratio on the GRSI performance

213 The DT γ /DTn branching ratio $\text{BR} = 2.07 \cdot 10^{-6}$ used for the GRSI design has been initially
 214 extrapolated from literature [5], as mentioned in Section 2.1. More recent observations and
 215 measurements of DT γ at OMEGA and NIF [12] provided an accurate estimate of BR equal to
 216 $4.20 \cdot 10^{-5}$, 20 time larger than the previous conservative assumption of $2.07 \cdot 10^{-6}$ with a big
 217 impact on the GRSI overall performance (Figure 6) [5][6].



218
 219 **Figure 6.** Comparison of the DTn cross section (black dashed line) with the derived DT γ cross sections
 220 obtained for BR values of $2.07 \cdot 10^{-6}$ (blue dotted line) and of $4.20 \cdot 10^{-5}$ (red line).

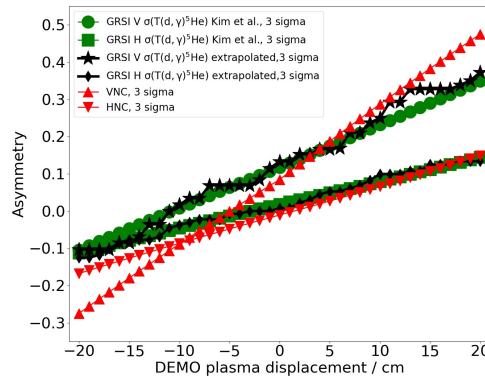
221 With this new BR value, the evaluation of the GRSI performance for DEMO plasma position
 222 control have been reassessed for different RNC geometry, i.e., collimator diameters,
 223 configurations [4]: Figure 7 shows GRSI DT γ count rates above 10 MeV in range of 0.01-7.50
 224 kCounts/s for 7.0 cm diameter collimator. If 9.0 cm diameter collimator is considered, the count
 225 rates result about 30-40 % larger.



226
 227 **Figure 7.** Comparison of GRSI simulated DT γ rates (vertical channels 1-12 on the left; horizontal
 228 channels 1-13 on the right) of energy events above 10 MeV for different RNC configurations: flux 0 and
 229 1; flux 0 and 1 power; flux 0 and 1 energy [4]. Input parameters $\text{BR} = 4.20 \cdot 10^{-5}$, 7.0 cm diameter
 230 collimator, 120.0 cm long LiH neutron attenuators and 3"x6" LaBr $_3$ (Ce) scintillator are assumed.

231 The GRSI as the RNC, being a multi-line of sight instrument with about 20 cm space resolution,
 232 can make use of the correlation of the individual measurements along each channel/detector to
 233 verify the possibility of control of DEMO plasma positioning. Considering RNC collimator
 234 geometry and the neutron fluxes [13], an asymmetry parameter [3] can be defined as

235 $Asymmetry = \frac{\sum_{i=1}^{n/2} N' - \sum_{i=\frac{n}{2}+1}^n N'}{\sum_{i=1}^n N'}$ with N' the measured number of counts within 10 ms
 236 (neutron for the HNC or VNC and γ rays for the GRSI) and n the number of lines of sight (12
 237 for the vertical cameras VNC and GRSI, and 13 for the horizontal HNC and GRSI). The
 238 Asymmetry has been studied for both RNC and GRSI for DEMO plasma displacements in the
 239 range [-20, +20] cm with 1 cm step vertically and horizontally. A linear relationship has been
 240 assumed for the measured counts as function of the displacements such that for a vertical
 241 displacement of -20 cm towards the bottom of the vessel, the top channel of the horizontal
 242 cameras would measure 0 counts, the second from the top would measure the same counts as for
 243 the top one for 0 cm displacement, and the last detector at the bottom would measure the same
 244 counts as the previous detector with 0 cm displacement. The linearity allows to consider the
 245 fraction of counts each detector measures as the ratio of the displacement with respect to the full
 246 displacement of +20 cm or -20 cm. Moreover, the effects of displacements are independent on
 247 the two cameras such that a vertical displacement impacts only on the horizontal cameras and
 248 not on the vertical cameras although DEMO plasma is closer to or farer from it. The GRSI
 249 Asymmetry green and black trends in Figure 8 for 4.20 cm diameter collimators illustrate how
 250 the GRSI performance strongly depends on BR and how the results look similar (green) to the
 251 ones of RNC (red) when adopting $BR = 4.20 \cdot 10^{-5}$ [12]. The GRSI Asymmetry trend for the
 252 conservative extrapolated $BR = 2.07 \cdot 10^{-6}$ (black) improves and becomes more similar to the
 253 green trend of Figure 8 if larger GRSI collimators are implemented, for instance, already with
 254 6.20 cm diameter.



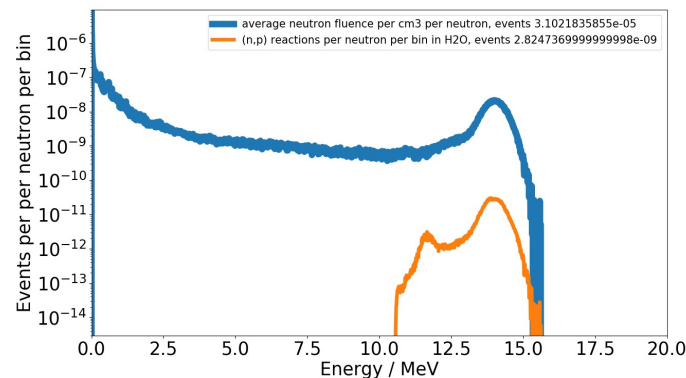
255 **Figure 8.** Comparison of the GRSI capability of detecting position variations with 1 cm spatial resolution
 256 of DEMO plasmas for $BR = 4.20 \cdot 10^{-5}$ (circle and square green markers for vertical and horizontal GRSI)
 257 with respect to the extrapolated $BR = 2.07 \cdot 10^{-6}$ previously used for the GRSI design (star and diamond
 258 black markers for vertical and horizontal GRSI). Both results are obtained for GRSI detector collimator of
 259 4.20 cm diameter and compared with RNC Asymmetry trend (up and down triangle red markers for VNC
 260 and HNC).
 261

262 As from the GRSI results displayed in Figure 8, although the 20 cm space resolution of the
 263 RNC-GRSI instrument, provided GRSI reaches reasonable count rates within 10 ms, which
 264 depends on RNC and GRSI collimator diameters, the Asymmetry parameter allows the GRSI
 265 (as well as the RNC) for the diagnosis of DEMO DT plasmas displacements as low as 1 cm,
 266 namely, making GRSI also theoretically capable of monitoring DEMO DT plasmas with 1 cm
 267 space resolution with 10 ms time resolution.

268 2.7 GRSI capability of assessing DEMO DT fusion power with gamma-ray measurements

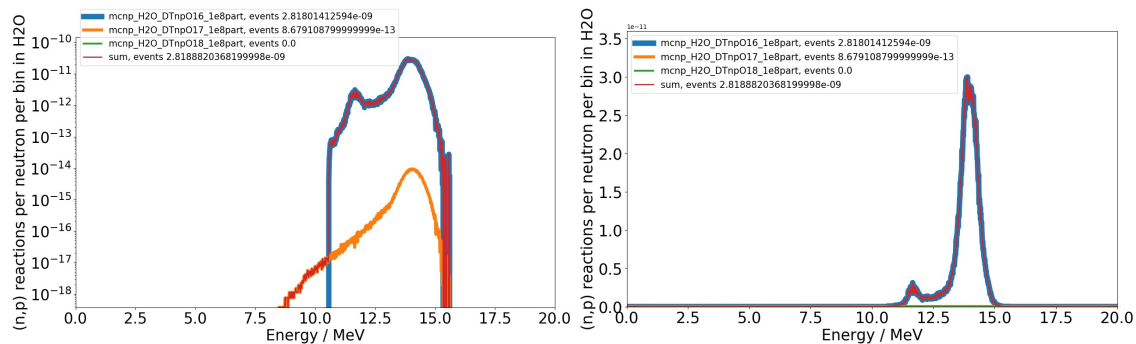
269 DT γ diagnosis constitutes an independent method with respect to DTn to assess DEMO DT
 270 nuclear fusion power and, as previously noted, it is almost background-free. Another way to

271 assess the fusion power from γ emission is to implement in DEMO the same method in use in
 272 nuclear fission reactors, namely, neutron activation of cooling water. In this case, the neutron
 273 activation reaction of interest is $^{16}\text{O}(n,p)^{16}\text{N}$ which features a Q value of 10.42 MeV. As formed,
 274 ^{16}N β^- decays with 100% branching ratio to either ^{16}O ground state or to one of its excited
 275 levels. In the latter case, γ emission to ^{16}O ground state happens with half-life of 7.14 s.
 276 Depending on the ^{16}O excited level, γ rays of 6.13 MeV (67% branching ratio) or 7.12 MeV
 277 (5%) are emitted [14]. The cross section of $^{16}\text{O}(n,p)^{16}\text{N}$ reaction is peaked about 11.65 MeV
 278 where it measures 0.115 b. For the energy of interest, namely DTn 14.05 MeV, the cross section
 279 is 0.043 b which is 37.4 % lower than the peak value [15][16][17][18].
 280 The investigation has been carried out with MCNP calculations of DEMO divertor
 281 [6][8][19][20][21]. As presented in Figure 3 (Section 2.2), DTn induced background in
 282 $\text{LaBr}_3(\text{Ce})$ γ spectrometer would make this γ emission hardly detectable if the spectrometer
 283 would be installed inside DEMO biological shielding. For this reason, the effect of water flow
 284 (10 m/s) and decay time (7.14 s half-life) are included in the assessment of the γ rays detectable
 285 outside the biological shielding of DEMO together with the selection of the best spectrometer
 286 for this task. The MCNP calculations have helped assess the (n,p) activation rates induced by
 287 DTn in DEMO cooling water: Oxygen isotopes ^{16}O (99.762 %), ^{17}O (0.038 %) and ^{18}O (0.2 %)
 288 are considered in the water composition. Figure 9 presents the (n,p) reaction production per
 289 neutron energy relative to the average of DTn fluence passing through the divertor (stainless
 290 steel block with a 2.035 cm layer of Tungsten facing the plasma) and reaching a 40.0 cm long
 291 and 6.41 cm radius water pipe made of EUROFER located underneath.



292
 293 **Figure 9.** Comparison of the average 14 MeV DTn flux through the water pipe (thick blue line) and the
 294 (n,p) reaction production per neutron energy (thin orange line) in log scale.

295 The (n,p) reaction production per neutron energy for the different isotopes of Oxygen in water is
 296 compared in Figure 10 according to their molecular abundance: ^{18}O has no reaction channel
 297 (n,p). As mentioned above, $^{16}\text{O}(n,p)$ reactions in water result in ^{16}N which decays with a half-
 298 life of 7.14 s emitting γ rays of 6.13 MeV (67 % branching ratio) and of 7.12 MeV (5 %).



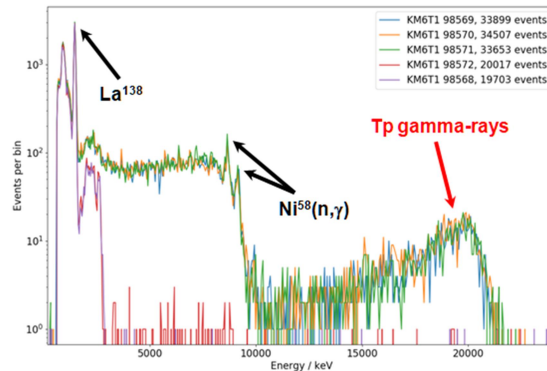
299
 300 **Figure 10.** Comparison of the (n,p) reaction production per neutron energy in ^{16}O (first thick blue line
 301 from the top), ^{17}O (second orange line) and ^{18}O (none, third thin green line) and their sum (fourth thin red
 302 line) in log (left) and linear (right, times $1 \cdot 10^{-11}$) scales.

303 To assess the ^{16}N production in the water cooling pipe, different DEMO power fluxes are
 304 considered [6][22]. As an example, DEMO 1.12 W/m^2 corresponds to $4.99 \cdot 10^{13}$ 14 MeV
 305 neutrons per second per cm^2 which, according to MCNP calculations, give rise to $1.41 \cdot 10^5$
 306 $^{16}\text{O}(n,p)^{16}\text{N}$ reactions per second per cm with the production of $9.42 \cdot 10^4$ 6.13 MeV (67 %) and
 307 of $6.89 \cdot 10^3$ 7.12 MeV (5 %) γ rays per cm . If now a 50 m long pipeline is considered to reach
 308 the measurement position outside DEMO biological shielding, the measurement can happen
 309 with a delay of 20.75 s which gives rise to $1.26 \cdot 10^4$ 6.13 MeV and to $9.19 \cdot 10^2$ 7.12 MeV γ rays
 310 per second per cm . Just for comparison to this γ emission distributed in 4π , a commercial γ
 311 source used as reference in laboratories usually features 37 kBq (37×10^3 decays per second).
 312 This confirms the possibility to design a specific γ detection system still based on $\text{LaBr}_3(\text{Ce})$
 313 detector technology to be installed outside DEMO biological shielding and capable to assess
 314 DTn fusion power produced in DEMO similarly to what in use in nuclear fission reactors.

315 2.8 GRSI capability of assessing the nT/nD ratio

316 As presented in Figure 3 (Section 2.2), GRSI measured spectrum above 10 MeV is only relative
 317 to $\text{DT}\gamma$ reactions in DEMO DT plasmas. Each $\text{DT}\gamma$ measured above 10 MeV corresponds to a
 318 triton burnt in the **DT** nuclear fusion process. Thus, with an accurate knowledge of $\text{DT}\gamma/\text{DTn}$
 319 BR, absolute calibrated GRSI γ spectrometers and detailed modelling and calculations of the
 320 DTn induced γ background in the measured $\text{DT}\gamma$ spectrum, the clear signature $\text{DT}\gamma$ events above
 321 10 MeV can be used to determine the amount of burnt T ions and contribute to the assessment
 322 of the nT/nD ratio for DEMO [23][24].

323 As a demonstration of the feasibility of high-energy $\text{DT}\gamma$ measurements with commercial
 324 $\text{LaBr}_3(\text{Ce})$ scintillator, it is worth to mention the experience matured at JET where two high rate
 325 γ $\text{LaBr}_3(\text{Ce})$ spectrometers (3"x6") are installed with vertical-radial and equatorial-tangential
 326 lines of sight. Recently, pure T plasma experiments have been carried out. Figure 11 shows the
 327 **19.81 MeV** $\text{T}\gamma$ spectra the spectrometer with equatorial-tangential viewing line measured at
 328 JET.



329
 330 **Figure 11.** Comparison in log scale of measured $Tp\gamma$ by a 3"x6" $LaBr_3(Ce)$ spectrometer at JET during
 331 the C39 TT experimental campaign on 12th December 2020 with 96 % T, 4 % H and 4 MW Ion Cyclotron
 332 Resonance Heating. The ^{138}La and ^{58}Ni peaks are used as references for calibrating the energy scale of the
 333 measured energy spectrum. The $LaBr_3(Ce)$ crystal features an intrinsic background radiation up to 3000
 334 keV corresponding here to the spectra measured for shots 98572 and 98568 for which $Tp\gamma$ emission was
 335 null.

336
 337 This result confirms the MCNP calculations and modelling of the reactions of interest in GRSI
 338 design and looks very promising for DT γ spectroscopy of DT fusion plasmas with commercial
 339 $LaBr_3(Ce)$ spectrometers both at ITER and for DEMO [11][5].

340 **3. Discussion**

341 In general, a diagnostic instrument benefits from direct viewing of the **radiation** source. In case
 342 of GRSI, it suffers not only from the constraints related to the RNC collimator geometry but
 343 also from the shielding effects of the RNC neutron detectors positioned along the GRSI lines of
 344 sight. Careful analysis needs to be performed to assess the impact on the DT γ fluxes when the
 345 geometry and materials of the **RNC** neutron detectors will be finalized. **However, this study has**
 346 **demonstrated the capability of GRSI to control and monitor DEMO plasma positions with the**
 347 **combination of count rates measured along all GRSI lines of sight by the Asymmetry parameter,**
 348 **as for the RNC. The Asymmetry analysis makes GRSI theoretically capable of monitoring**
 349 **DEMO DT plasmas with 1 cm space resolution with 10 ms time resolution, provided GRSI**
 350 **reaches reasonable count rates within 10 ms, which mostly depends on RNC and GRSI**
 351 **collimator diameters.**

352 The DTn induced γ background is in principle 4π distributed and affects the full volume of the
 353 GRSI spectrometer crystal. To reduce its impact on the DT γ measurements, the diameter of the
 354 GRSI collimators, **filled with LiH neutron attenuators**, at the back of the HNC-VNC neutron
 355 detector boxes should be preferably wider than the GRSI detector diameter: To maximize the
 356 detector signal-to-background ratio is best to expose its full volume to the direct incoming
 357 radiation. For these reasons as well as for possible constraints induced by the HNC-VNC
 358 geometry configurations, GRSI collimators and LiH diameters of 9.0 cm, wider than the 3"
 359 (7.62 cm) diameter of the GRSI crystal, should be considered. More detailed MCNP
 360 calculations will be required when the RNC-GRSI shielding material will be chosen. However,
 361 given the surface-to-volume ratio of DEMO DT plasmas and the GRSI geometry, the GRSI can
 362 provide equivalent diagnostic information with **only 7** vertical and horizontal lines of sight with
 363 respect to the 12 and 13 of VNC and HNC.

364 **Concerning GRSI capability of diagnosis of DT plasma impurities and assessing their**
 365 **concentration via X-ray emission spectroscopy, a specific instrument is recommended as well as**
 366 **for specific DT power measurements. Here, as performed in nuclear fission reactors, DEMO can**

367 be equipped with a GRSI similar 3"x6" LaBr₃(Ce) scintillator installed along the cooling water
368 pipeline outside DEMO biological shielding.
369 The progress on the GRSI conceptual studies presented in this article shows the relevant
370 potential of this diagnostic to contribute to DEMO plasma positioning control with time
371 resolutions which depend on the RNC and GRSI collimator diameters and on the BR value. The
372 GRSI can back up the RNC diagnostic information but being also capable of assessing
373 independently both DEMO DT fusion power and T burning fraction from its almost
374 background-free DT γ measurements above 10 MeV. Moreover, being so remotely installed and
375 protected by the LiH neutron attenuators, the GRSI should not suffer the intense and prolonged
376 DTn fluxes which impact on the materials structure. The LiH neutron attenuators will suppress
377 the neutron direct radiation complying with the dose limits defined for DEMO outside its
378 biological shielding.
379 The GRSI conceptual studies will soon enter a new phase of the DEMO diagnostics (sensors)
380 definition, with efforts devoted to the engineering of the instrument.

381 Acknowledgments

382 This work has been carried out within the framework of the EUROfusion Consortium and has
383 received funding from the Euratom research and training programme 2014-2018 and 2019-2020
384 under grant agreement No 633053. The views and opinions expressed herein do not necessarily
385 reflect those of the European Commission.

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