Comparing the bulk radiated power efficiency in carbon and ITER-like-wall environments in JET.

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Abstract

We use a parameter \( \beta_r \) for all plasmas that allows detecting the pollution of the plasma bulk by highly radiative impurities. This parameter is defined as the radiative loss of the mixture of impurities relative to their mean \( Z^2 \) and was used in previous works to characterize the efficiency of radiative mantles in Neon seeded discharges [1,2]. We show that this parameter, though global, is very sensitive to the presence of highly radiative impurities in the bulk of the discharge. We use it to compare JET plasmas in the carbon environment and in the ITER-like wall (ILW), where it is highly correlated to the level of a bundle of spectroscopic lines of tungsten passing through the center of the discharge. In the carbon environment, the value of \( \beta_r \) is around \( 10^{-40} \text{ MW.m}^6 \), indicating the absence of highly radiative impurities in the plasma. No change or even a small decrease is observed when going from L-mode to H-mode, this robustness being in agreement with the multi-machine scaling [3]. In the ILW machine, the value of \( \beta_r \) is found to depend on the type of additional heating and confinement state of the plasma. We observe that neutral-beam injection (NBI) introduces little W into the plasma, with a \( \beta_r \) between 2 and 3 \( 10^{-40} \text{ MW.m}^6 \). Ion-cyclotron radio-frequency (ICRF) waves yield a \( \beta_r \) of order 5 in L-mode and 10 \( 10^{-40} \text{ MW.m}^6 \) in H-mode when no edge-localized modes (ELMs) are present. Conversely when ELMs are present, the parameter goes back to 5 \( 10^{-40} \text{ MW.m}^6 \), illustrating the positive effect they can have on the bulk pollution by tungsten.

1) Introduction

Currently tungsten is being investigated as the preferred future material for tokamak divertors as its use solves several problems:
Firstly fuel retention should be minimized by the use of tungsten [4,5]. Another positive aspect is that tungsten is eroded at higher plasma temperatures than carbon [6]. At the moment it is being tested in a number of tokamaks [7] and is envisaged as the primary material for the divertor of ITER. However, apart from these qualities, there are some difficulties to operate machines with this type of high-Z material, in particular because it can

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radiate in the plasma core [8,9]. The presence of radiation in the centre of the discharge is a major concern for fusion machines and efforts are being made in present experiments to identify the radiating impurities in the core and estimate their concentration [10,11].

Spectrometers used for this purpose cover only lines or bundles of lines of these high-Z impurities along a few lines-of-sight and it is difficult from these measurements to deduce their concentrations. This can be done, in principle, with the help of radiation models which calculate the ionization equilibrium and which must include also the transport of the impurities. For high-Z elements, however, even the best of these models are only approximate. The case of tungsten is particularly difficult since many coefficients of radiative transitions in the highest ionization states are presently unknown.

As a result any information that can be gained about the radiative efficiency of the impurities polluting the bulk can be useful. This paper is a step in that direction. We analyze data from JET in its ITER-like wall (ILW) configuration [12] with tungsten divertor and beryllium torus walls and compare it with similar data from the earlier carbon environment.

We show that even global parameters can carry useful information about the type of impurities polluting the bulk of the discharge. In order to do this, we use a parameter defined in previous works [1,2] as the quality of impurity cooling. We show that this parameter is in fact the radiative loss parameter of the mixture of impurities relative to the square of their mean charge. The value and the changes of this parameter (called $\beta_r$) indicate a change of the impurity mixture and the pollution of the plasma by highly radiative species becomes immediately visible regardless of the level of the bulk radiated power.

In section 2 we write $\beta_r$ as a function of $P_{\text{rad bulk}}$, $Z_{\text{eff}}$ and $n_e$ and give its units. In section 3, the physical meaning of $\beta_r$ is discussed in the light of the “radiative loss parameter” of the impurities as defined in reference [13]. In section 4, we discuss the filtering and influence of ELMs on determination of this parameter. Section 5 is dedicated to carbon-environment plasmas where we observe the changes in the $\beta$ parameter from L to H mode. Section 6 is dedicated to the results obtained in the ILW environment. In 6.a we monitor first the different effects of NBI and ICRF waves on $\beta_r$ in L-mode plasmas. In 6.b and 6.c we observe the L-H transition both for pure NBI and ICRF plasmas. In 6.d, we observe the correlation of $\beta_r$ with the spectroscopic lines of high-Z impurities and discuss the effect of tungsten events on the evolution of the parameter. In section 7 we discuss how the different values of $\beta_r$ affect the validity of the multi-machine scaling. We finish in section 8 with the conclusions.

2) The definition of $\beta_r$.

Following references [1,2] a parameter characterizing the quality of cooling of the impurities in the bulk of the plasma can be written as:

$$\beta_r = P_{\text{rad bulk}} / (Z_{\text{eff}} - 1) \ n_e^2$$  \hspace{1cm} (1)

where $n_e$ is the line-averaged density provided by high-resolution Thomson scattering [14] and $Z_{\text{eff}}$ is calculated from bremsstrahlung emission [15] measured along a horizontal line-of-sight crossing the plasma centre (i.e. not passing through the divertor region). $P_{\text{rad}}$ in the bulk is evaluated by bolometry, on JET using its most recent cameras (KB5) [16]. In order to estimate the bulk radiated power, only bolometric horizontal lines-of-sight in the top half of
the plasma are used to avoid the divertor region. The volume covered by them is then stretched to fill the whole plasma volume. In JET, we neglect the bremsstrahlung and cyclotron radiation; however, in a machine such as ITER, they will significantly contribute to the total radiated power and will have to be calculated or measured and removed from the total radiated power before evaluating $\beta_r$.

We give the expression for $\beta_r$ in a general case where different types of impurities with different ionization levels are present in a deuterium plasma.

Firstly, for only one type of impurity, the radiated power (W) in the bulk may be written as:

$$P_{\text{radbulk}} = n_e n_{\text{imp}} V \sum_i a_i b_i c_i L_{i;i}.$$  

In this expression $n_{\text{imp}}$ is the total impurity density, $n_e$ the electron density, $a_i= n_i/n_{\text{imp}}$ the fraction of impurity ions with charge $Z_i$, $b_i= n_{ei}/n_e$, the fraction of the density in the volume where the ion with charge $Z_i$ radiates and $c_i=V_i/V$ the fraction of the volume in which the same ion radiates. $L_{i;i}$ is the radiative cooling function for the same ion with charge $Z_i$. We suppose in this expression that the electron temperature $T_e$ is homogenous in the volume where the ion of charge $Z_i$ is radiating, as well as the density of impurity ions and electrons. This is only an approximation. If we extend this to multiple impurities we get:

$$P_{\text{radbulk}} = n_e V \sum_k n_{\text{imp}}^k \sum_i a_i^k b_i^k c_i^k L_{i;i}^k,$$

where $k$ denotes different types of impurities. We can write the expression for $Z_{\text{eff}}$ as:

$$Z_{\text{eff}} = 1 + \frac{1}{n_e} \sum_k n_{\text{imp}}^k \sum_i a_i^k Z_i^k (Z_i^k - 1).$$

If we combine these expressions in relation (1), we find

![Figure 1](shot_no_76676.png)

Shot 76676, carbon environment. H-mode, 12MW NBI heating. The result displayed is for the NBI heating phase only.

Left) $\beta_r$ determined with ELM-filtered data. Right) $\beta_r$ determined with unfiltered data.
\[
\frac{\beta_r}{V} = \frac{\sum_k e_{\text{imp}}^k \sum_i a_i^k b_i^k c_i^k L_{ii}^k}{\sum_k e_{\text{imp}}^k \sum_i a_i^k Z_i^k (Z_i^k - 1)} \quad \text{(2)}
\]
where \( e_{\text{imp}}^k = \frac{n_{\text{imp}}^k}{\sum_k n_{\text{imp}}^k} \).

We notice that \( \beta_r/V \) is not dimensionless; it has the dimension of \( L_t \) and can be expressed in \( \text{W.m}^3 \).

3) The physical meaning of \( \beta_r \)

The radiative loss parameter of an impurity \( k \) is defined in reference [13] as
\[
S_k = \frac{\hat{P}_{\text{rad}}}{(n_e n_k)} ,
\]
where \( \hat{P}_{\text{rad}} \) is radiated power density (\( \text{W/m}^3 \)). It can be calculated using the same notations used in section 2.

We find in this case that for an impurity \( k \),
\[
S_k = \sum_i a_i^k b_i^k c_i^k L_{ii} ,
\]
where \( i \) takes account of the different ionization states.

Hence \( \beta_r \) can be expressed as a function of the radiative loss parameter of the impurities as:
\[
\frac{\beta_r}{V} \approx \frac{\sum_k e_{\text{imp}}^k S_k}{\sum_k e_{\text{imp}}^k \sum_i a_i^k Z_i^k (Z_i^k - 1)} . \quad \text{(3)}
\]

\( \beta_r \) is the weighted sum of the radiative loss parameters of the different impurities divided by the weighted sum of their average \( Z^2 \). In reference [13], \( S_k \) has been calculated for different types of impurities in the bulk of the ASDEX Upgrade plasma using a coronal model. The coronal approximation can be used if the residence time of the impurities in the plasma is sufficiently long and the plasma in steady state. These conditions can be encountered in the plasma bulk in contrast to the SOL and divertor region where the residence time of the impurities is too short. The result does not depend strongly on the plasma density as long as multi-step processes are weak. If we take the two main impurities for the JET shots (carbon

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**Figure 2**

Left: scenario for shot 75911, L-mode, carbon environment, NBI from 1.4 to 2.8 MW

Right: \( \beta_r \) calculated during the NBI heating phase.
for the carbon divertor and tungsten for the ILW configuration), the results shown in Figure 1b of reference [13] show that the $S_W$ for tungsten has a value of order $10^{-31}$ Wm$^3$ while it is less than $10^{-34}$ Wm$^3$ for carbon. For W, the radiative loss parameter increases moderately for $T_e$ above 100 eV (about a factor of 2 between 100 eV and 3 keV), while it decreases by a factor of 10 for carbon in the same range. Tungsten is at least 3 orders of magnitude more efficient at radiating in the bulk than carbon. This is the well-known result that a tiny fraction of tungsten (~$10^{-4}$) can drive a large radiation loss in the bulk.

The changes of $\beta_r$ may be discussed using the dependence of the radiative loss parameter with $T_e$ discussed above. In the case of W pollution, the rather weak dependence of $S_W$ with $T_e$ leads to the conclusion that an increase of $\beta_r$ must be associated with an increase of the relative concentration of W in the bulk $\varepsilon_{\text{imp}}^W$ (even if this increase is not sufficient to have a measurable impact on $Z_{\text{eff}}$). The second point is that the value of $\beta_r$ is liable to be very resilient to $T_e$ changes with tungsten pollution. A decrease of $\beta_r$ can have different causes. It can be triggered by an increase of low $Z$ impurities that have a very small radiative loss parameter (increase of $Z_{\text{eff}}$ and of the denominator in relation (2)), or a decrease of the fraction of tungsten impurities $\varepsilon_{\text{imp}}^W$ (no or small change in $Z_{\text{eff}}$), or a combination of these two processes, as is occurring during impurity seeding, for example. Finally, in the ILW changes in the values of $\beta_r$ will always indicate a change in the bulk impurity mixture.

4) Experimental determination of $\beta_r$ filtering

$\beta_r$ can be simply calculated as:

$$\beta_r(t) = \frac{P_{\text{rad bulk}}(t)}{(Z_{\text{eff}}(t) - 1) n_e^2(t)}.$$ 

The value of $\beta_r$ is interesting to follow during plasma shot to check rapidly if the mixture of impurities contaminating the bulk evolves. However strong events such as ELMs [17] may
affect relation (1) or even break it, so that it is in principle necessary to filter the data to
remove these non-stationary phases before computing $\beta_r$. However we show that from an
operational point of view, $\beta_r$ can be calculated without filtering. There are two reasons for
this: $P_{\text{rad bulk}}$ is quite insensitive to ELMs that are edge phenomena, and $Z_{\text{eff}}$ and $n_e$ are line-
averaged quantities rather resilient to edge changes. In order to illustrate this, we show in
Figure 1 $\beta_r(t)$ calculated for a 12 MW NBI H-mode shot with very large Type I ELMs where
first a histogram filtering is applied on the ELMs (Fig.1, left) and second without ELM
filtering (Fig.1, right). This comparison shows that even without filtering, the value of $\beta_r$ can
be reliably recovered and followed in time.

5) Carbon environment L and H-mode.

The results which we present are selected from the whole JET database and several thousand
shots have been scanned automatically. Although only the results from a few shots are
presented here, they do represent very well the behavior of the whole data base. In Figure 2 $\beta_r$
is plotted for an L-mode shot with low NBI power (1.4 MW). It is found to be very close to 1
throughout the heating phase. In Figure 3, the same parameter is plotted for an H-mode shot
with ELMs. Two heating phases are present, the first with 9 MW of NBI power, the second
with 19 MW of NBI power. Figure 3 shows that that the 9 MW phase has a value of $\beta_r$ close
to $10^{-40}$ MW.m$^6$ as in L-mode. At 19 MW, $\beta_r$ decreases to $0.5 \times 10^{-40}$ MW.m$^6$. This decrease
may be attributed to the behavior of the radiative loss parameter of the light impurities and
carbon ($S_C$) in particular. As the additional heating power is increased, $T_e$ increases in the
whole bulk including the pedestal region. As a consequence, the radiative loss parameter of
carbon decreases (numerator of relation (2)), thus yielding lower $\beta_r$ values. This simply
indicates that the low-Z impurities become even less efficient at radiating in the bulk when $T_e$
is increased. We notice that at NBI heating power below 10 MW, the $\beta_r$ value is the same in L
or H-mode. This shows how robust the value of the $\beta_r$ parameter is in carbon discharges, an
observation which partly explains the robustness of the multi-machine scaling [3].

6) ITER-like-wall environment
6.a Specific effects of ICRF and NBI heating in L-mode

In the carbon environment, wall and divertor surfaces are eroded by physical and chemical erosion but there are also specific interactions of the additional heating with the walls. In the ILW environment, these specific interactions with the wall and divertor structure are critical and the level of tungsten sent to the bulk of the discharge may depend on the type of additional heating. This is examined by comparing ICRF and NBI heating at same power during stationary L-mode conditions. The scenario of the shot plotted in Figure 4 shows that during ICRF heating $Z_{\text{eff}}$ is larger than during NBI heating. This suggests that during ICRF more low-Z impurities are also released (such as Be) than during NBI heating, suggesting stronger interaction with the walls. However, the value of $\beta_r$ is $7 \times 10^{-40}$ MW.m$^6$ during ICRF and $3 \times 10^{-40}$ MW.m$^6$ during NBI heating. This clearly shows that during ICRF the bulk plasma is more polluted by highly radiative impurities than during NBI operation.

If we compare these results with those obtained in the carbon environment, we see that during NBI heating in the ILW, the lowest value of $\beta_r$ is somewhere between 2 and $3 \times 10^{-40}$ MW.m$^6$, while the average value in carbon is $10^{-40}$ MW.m$^6$. This higher value suggests that even with NBI heating some small amount of tungsten is nevertheless added to the bulk.

6.b ILW transition from L- to H-mode with NBI only.

Figure 5 illustrates the behavior of $\beta_r$ when there is an L-H transition with NBI heating. We notice first that the value of $\beta_r$ during the L-mode phase ($8s < t < 10s$) is lower than that measured in L mode in the shot above (#81855), 2.2 instead of $3 \times 10^{-40}$ MW.m$^6$. The pollution of the plasma by high-Z impurities depends on its history and using NBI after ICRF is less favorable than operating with NBI heating only. We have measured values as low as $1.3 \times 10^{-40}$ MW.m$^6$ for some ILW plasmas during NBI heating, close to those measured in carbon pulses. During the H-mode phase ($10s < t < 14s$), $\beta_r$ increases from 2.2 to an average $3.7 \times 10^{-40}$ MW.m$^6$, a 68% increase, though the additional heating power is increased from 1.5 MW to 10 MW. This result illustrates the fact that in this scenario, NBI heating increases moderately the amount of
high-Z impurities from L to H-mode. The fact that $Z_{\text{eff}}$ remains unchanged also indicates that the pollution by low-Z impurities does not increase either.

6.c ILW transition from L to H-mode with ICRF only

In Figure 6, a plasma where only ICRF is used triggers an L-H transition. This is visible in the plasma traces where the Dalpha signal drops at $t=17.27s$. The energy stored in the discharge is not sufficient to trigger ELMs. This type of shot has been studied and reported in [18]. After the transition, $\beta_r$ increases from 5 to 10, a 100% increase. $\beta_r$ around $10^{-39}$ MW.m$^6$ is the level commonly observed in the JET database when ICRF triggers H-modes without ELMs. It is one of the highest values obtained so far for $\beta_r$. As the level of $\beta_r$ jumps to $10^{-39}$ MW.m$^6$ immediately after the ICRF power has reached the threshold, it can be speculated that this is partly the effect of the transport change. As transport decreases in the bulk after the H transition, the amount of impurities there increases. There are other indications of fluctuation effects on $\beta_r$ when ICRF power is being used. For example, when ELMs are present in the discharge, a combination of NBI and ICRF heating power produces $\beta_r$ values of order only $5 \times 10^{-40}$ MW.m$^6$. Once the NBI is decreased and the ELMs disappear, $\beta_r$ reverts to $10^{-39}$ MW.m$^6$. This illustrates the beneficial effect of ELMs for ejection of impurities from the bulk.

6.d Linkage of $\beta_r$ with impurity lines measured in the discharge.

$\beta_r$ is the radiative loss parameter of the mixture of impurities polluting the bulk relative to its mean $Z^2$. As the radiative loss functions of the impurities scale roughly as the cube of their charge [13], the $\beta_r$ values are extremely sensitive to the amount of highly radiative impurities such as tungsten in the discharge; thus $\beta_r$ is a potent parameter to detect them. As a result it can be expected that $\beta_r$ should be correlated with the spectroscopic lines of these highly radiative impurities [19]. This is indeed the case, as is shown in Figure 7. $\beta_r$ is plotted as a function of time together with a bundle of tungsten lines emitted from the center and a line of nickel XVIII radiated at the edge of the bulk plasma. To facilitate comparison of the different
signals, they are normalized to the same scale. The data in Figure 7 show that $\beta_r$ is extremely well correlated with the bundle of W lines between 8 and 9 s but shows no variation with the very large peak of nickel XVIII that occurs between 10 and 10.2 s. As a result, the increase of the bulk radiation at the beginning of the NBI heating phase where both tungsten and nickel lines are present (8 to 9 s) can be unambiguously attributed to emission by tungsten. Another point is that $\beta_r$ can detect a pollution of the bulk plasma after a so-called tungsten event, i.e. a sudden large influx typically due to ingestion of a dust grain or flake [20]. This is illustrated in Figure 8 which shows the variation of $\beta_r$ during a time window where such an event occurs in a 9.4 MW NBI heated H-mode shot. The ELMs have not been filtered and are visible on the data. It is unclear at the moment the exact role they play in the recovery time of the radiation profile which exactly matches the recovery time of the $\beta_r$ parameter. We note however a drop of the $\beta_r$ parameter at 13.5 s that corresponds to the crash of a very large ELM. Before the tungsten event, at 13.4 s, $\beta_r$ has a value of $2.6 \times 10^{-40}$ MW.m$^{-6}$, then after the event at 13.7 s, it goes back to a value around $4 \times 10^{-40}$ MW.m$^{-6}$, indicating that some tungsten has been added to the bulk of the discharge and is radiating there. This is confirmed by the bolometry data which show peaking of the radiation in the mid-plane on the low-field side of the plasma (the usual signature of tungsten radiation with NBI heating [21]) after the tungsten event. Notice also that before the tungsten event, the value of $\beta_r$ was quite low and in agreement with that usually observed with NBI heating.

7) $\beta_r$ and the multi-machine scaling.

The multi-machine scaling of plasma purity with radiated power [3] was derived at a time when most of the tokomaks involved were equipped with carbon wall components. The most common impurity in the plasma apart from carbon was oxygen, which is another low-Z impurity also radiating at low $T_e$. The scaling was used to characterize the total radiated power (bulk and divertor) in a situation where it was dominated by the radiation of low-Z impurities. Such a situation implied that most of the time, the radiated power in the divertor was larger than that from the bulk. The simplified form of the multi-machine scaling reads as:
\[ Z_{\text{eff}} \approx 1 + 7.5 \frac{P_{\text{rad}}^{\text{tot}}}{A n_e^2} \]  \hfill (4)

where \( P_{\text{rad}}^{\text{tot}} \) is the total radiated power and \( A \) the plasma envelope area.

First of all, the multi-machine scaling is not directly related to the value of \( \beta_r \) that we measure in the bulk of the discharge. This is because of the contribution of the divertor radiation to this scaling. We found in section 5 that \( \beta_r \) varies between 0.5 and \( 1.10^{-40} \) MW.m\(^6\) in carbon environment. For this range of values, the multi machine scaling is verified with some dispersion. This is shown in Figure 9 where the \( Z_{\text{eff}} \) predicted by the scaling is plotted versus the \( Z_{\text{eff}} \) integrated along a vertical Line of Sight. The result plotted in Figure 9 is for the shot displayed in Figure 3. In this shot, the bulk and divertor radiated power are about at the same level. However, most of the dispersion of the data around the scaling observed in Figure 9 comes from the contribution of the divertor radiation. This is a general result, we find that if most of the time the bulk radiation follows the generic form of the multi-machine scaling law very well (the points are well aligned i.e. \( \beta_r \) varies little), the divertor radiation has the correct order of magnitude to fit in the scaling but introduces some dispersion or even deviation from this scaling. This comes from the fact that the divertor radiation has less correlation with the changes of global parameters (like mean Ne and line integrated \( Z_{\text{eff}} \)) than the bulk radiated power.

In general, we find that for experimental values of \( \beta_r < 4 \times 10^{-40} \) MW.m\(^6\), there is reasonable agreement of the data with the multi-machine scaling. For rising \( \beta_r \) values above \( 4 \times 10^{-40} \) MW.m\(^6\), the data depart increasingly from the multi-machine scaling.
In order to illustrate this, we plot in Figure 10 left the prediction of $Z_{\text{eff}}$ by the multi-machine scaling as a function of $Z_{\text{eff}}$ issued from the experiment. The shot is #81855 in ILW studied in section 6.a and plotted in Figure 4. The $Z_{\text{eff}}$ predicted by the multi-machine scaling is systematically above the one calculated from experimental data. The difference between the $Z_{\text{eff}}$ predicted by the scaling and the one issued from the experimental data is plotted as a function of $\beta r$ in Figure 10 right. The data with a $\beta r$ around 3 (NBI phase) has a $Z_{\text{eff}}$ close to the one predicted by the multi-machine scaling but the data with $\beta r$ around 7 (ICRF phase) departs from the prediction of the scaling by about 1 (2.7 instead of 1.7). Figure 10 right clearly shows that the departure from the multi-machine scaling occurs with the increase of $\beta r$ i.e. when high Z impurity in the plasma bulk start to dominate the radiated power. Another way to measure the departure from the multi-machine scaling is to define a radiation enhancement factor:

$$D = \frac{P_{\text{rad}}^{\text{tot}}}{\frac{A}{Z^2} (T_{\text{eff}} - 1) N_{\text{Z}}^{2}}$$ (5)

This enhancement factor is a dimensionless parameter that measures how much more radiative the discharge is, relative to the situation where the multi-machine scaling applies. We plot D for the shot 81855 in Figure 11 together with $\beta r$. Although the two parameters measure different things ($\beta r$ measures the radiative efficiency of the bulk impurities relative to their mean $Z^2$ without reference to any scaling, D is an enhancement factor of the total radiated power relative to a scaling law) the behavior of the two parameters as a function of time is similar. Discharge #81855 is 2.5 five times more radiative than what predicted by the scaling during ICRF heating while it is only 1.5 more radiative than the scaling in the case of NBI heating. The comparison of the behavior of $\beta r$ and D in Figure 11 leads to the same conclusion than the one obtained from Figure 10 right: The deviation from the scaling comes from the radiation of high Z impurities in the bulk.

Figure 10

Left) shot 81855, ILW, ICRH and NBI heating, $Z_{\text{eff}}$ predicted by multi-machine scaling as a function of $Z_{\text{eff}}$ from experimental data (horizontal LOS Bremsstrahlung). Right) difference between $Z_{\text{eff}}$ from multi-machine scaling and $Z_{\text{eff}}$ from experimental data as a function of $\beta r$. 

$Z_{\text{eff}}$ predicted by the scaling and the one issued from the experimental data is plotted as a function of $\beta r$ in Figure 10 right. The data with a $\beta r$ around 3 (NBI phase) has a $Z_{\text{eff}}$ close to the one predicted by the multi-machine scaling but the data with $\beta r$ around 7 (ICRF phase) departs from the prediction of the scaling by about 1 (2.7 instead of 1.7). Figure 10 right clearly shows that the departure from the multi-machine scaling occurs with the increase of $\beta r$ i.e. when high Z impurity in the plasma bulk start to dominate the radiated power. Another way to measure the departure from the multi-machine scaling is to define a radiation enhancement factor:

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Finally all these results show that the multi-machine scaling corresponds to the lowest radiation state of the bulk plasma and does not apply when high-Z impurities start to dominate the radiation in that region.

8) Conclusions

We have shown that a global time-dependent parameter can give fast and reliable information about the presence of highly radiative impurities in tokamak discharges. It can be calculated for all plasmas regardless of the scenario and of the confinement state. We have compared JET shots in the carbon environment with ones in the ILW. The very low values obtained in the carbon environment clearly indicate the absence of significant radiation from highly radiative impurities in the bulk plasma, with an average value of $\beta_r \approx 10^{-40}$ MW.m$^6$. One of the striking results in carbon is that the variation of $\beta_r$ with the plasma scenarios is small. This result and the lack of high Z impurities in the bulk partly explain the robustness of the multi-machine scaling [3] that applies to carbon-dominated plasmas and proposes a constant coefficient of the scaling for each machine. When switching to the ILW environment, we observe that $\beta_r$ increases above $10^{-40}$ MW.m$^6$, this increase being clearly dependent on the plasma scenario: viz. the confinement mode, the fuelling of the plasma, and the type of additional heating used. By comparing the effect of the different types of additional heating, we observe that plasmas heated with NBI have relatively low $\beta_r$ values of order 2 to 3 times those measured in the carbon environment. This indicates that NBI heating sends little tungsten into the bulk. During the L-H transition the $\beta_r$ parameter is observed to increase moderately even with a sevenfold increase of the NBI power. This situation compares well with the carbon environment and confirms lower pollution by tungsten. We notice however that even with this type of additional heating, tungsten emission events do occur and can lead (in the case of strong events) to pollution of the core by additional tungsten. $\beta_r$ is strongly correlated to the bundle of W lines measured in the centre of the discharge but can still detect W pollution even when the radiation is displaced out of the spectroscopic lines-of-sight. In the case of ICRF heating, these plasmas systematically yield $\beta_r$ values of order $5 \times 10^{-40}$ MW.m$^6$ in L-mode and $10^{-39}$ MW.m$^6$ in H-mode if no ELMs are present. In the presence of ELMs, $\beta_r$ values are reduced back to a value of $5 \times 10^{-40}$ MW.m$^6$, illustrating very well the beneficial effect of ELMs on W pollution. In the ILW, we have not observed so far (experimental campaigns 2011, 2012, 2013) values of $\beta_r$ below $4 \times 10^{-40}$ MW.m$^6$ when ICRF heating.

![Figure 11](image.png)

Shot 81855, ILW, ICRH then NBI heating. D radiation enhancement factor and $\beta_r$ as a function of time.
power is being used to heat the plasma. These values are extremely robust over the entire database and clearly indicate that ICRF introduces more tungsten in the discharge than NBI heating.

Finally $\beta_r$ is a global parameter that is not straightforward but it has some advantages: it is extremely sensitive to the pollution of the plasma by highly radiative impurities and can detect them when no other diagnostic is available. It has also the virtue of being easily calculated with standard measurements and can be determined even without ELM filtering. Its time evolution can be followed during a shot and provides quantitative values which can be compared from shot to shot. For these reasons, it could be useful for many machines, including ITER.

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**References**

[1] G. Telesca et al, Nucl. Fusion, 40, No. 11,1845